

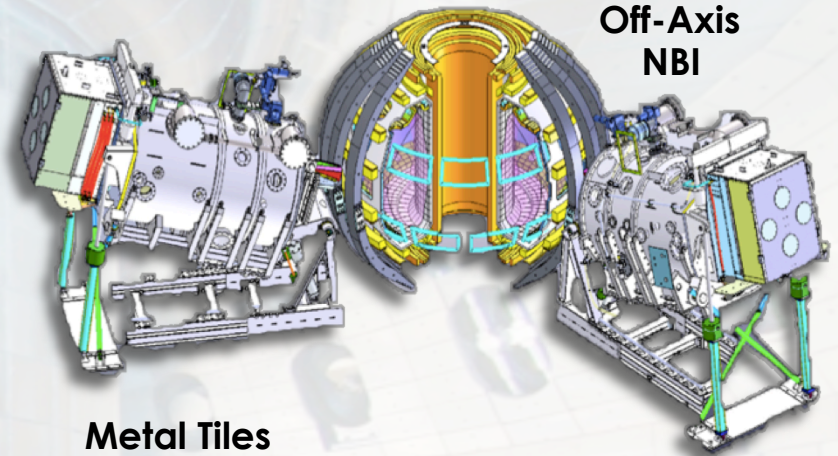
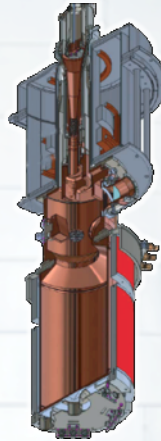
# DIII-D National Fusion Program Research Directions

by  
D.N. Hill

Presented to  
Fusion Energy Sciences  
Advisory Committee  
Bethesda, Maryland

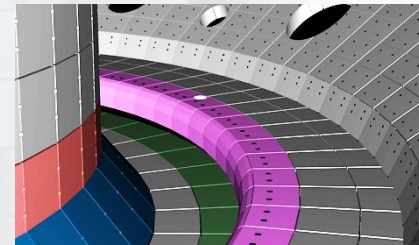
January 14, 2016

Electron  
Cyclotron

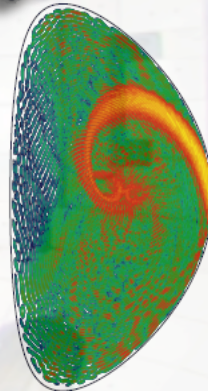


Off-Axis  
NBI

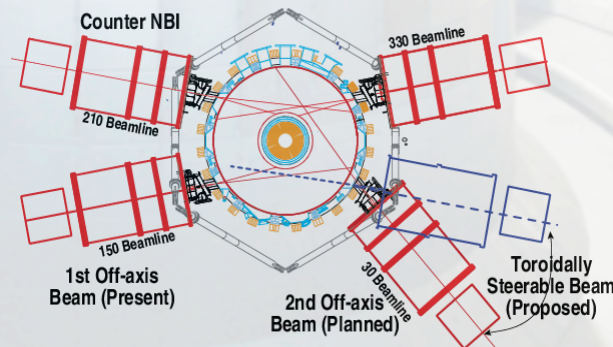
Metal Tiles



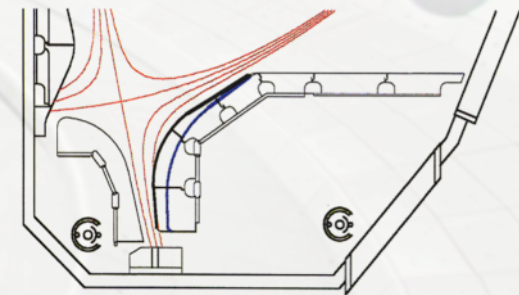
Helicon  
Current  
Drive



Co/Counter/Balanced NBI



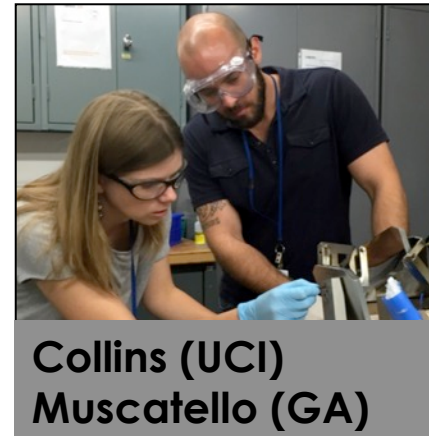
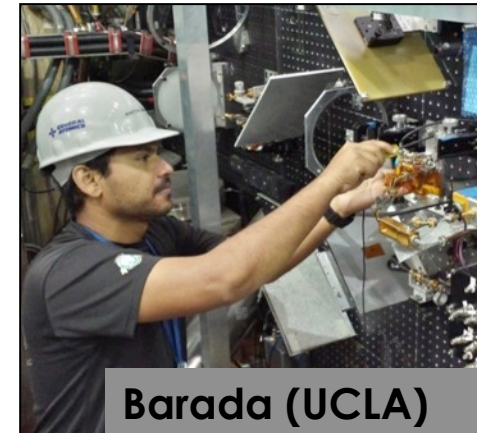
Divertor Shape Variation



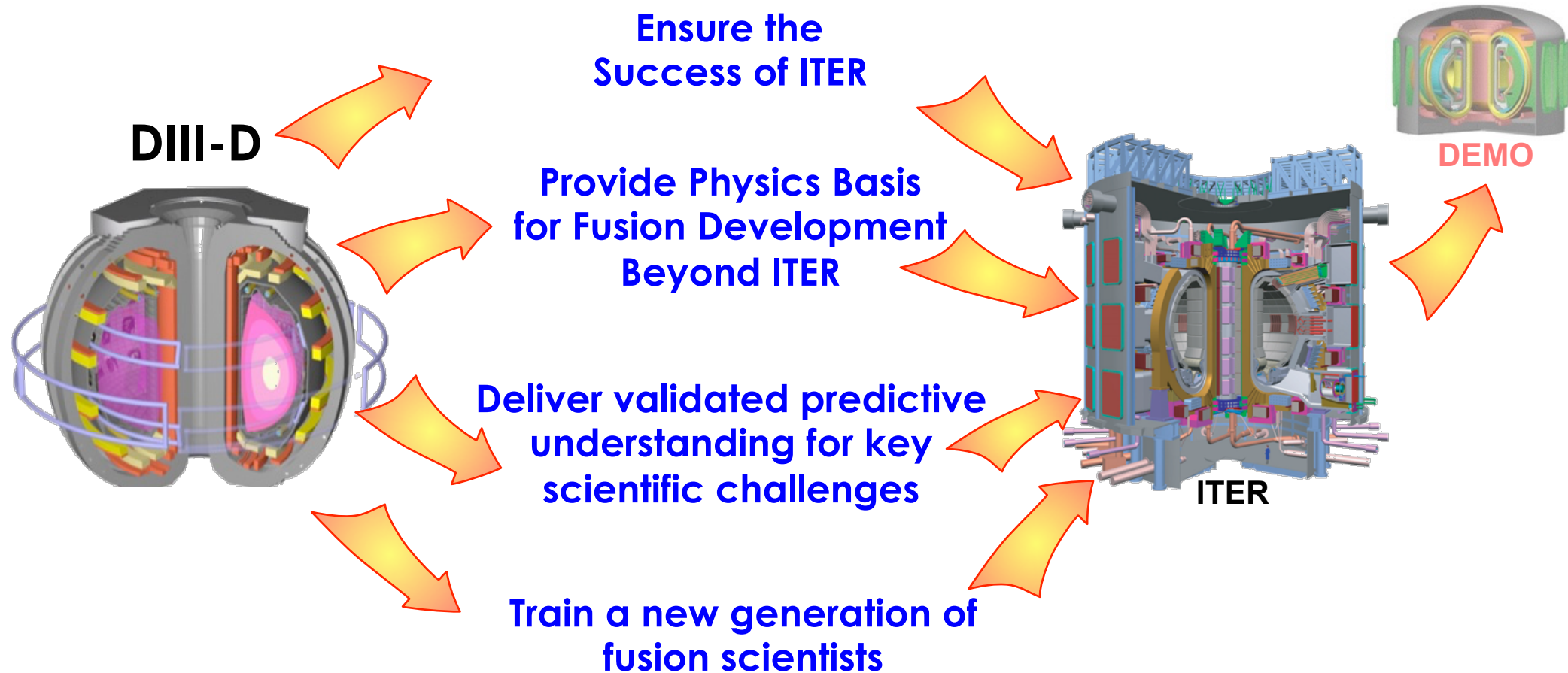
# Our Vision For The DIII-D Program is Based on Three Guiding Principles

- **Research With an Energy Goal**  
Research goals address challenges to achieving fusion energy
- **Scientific Excellence**  
Fastest route to success and developing predictive capability
- **World Class Facility for U.S. Office of Science**  
Upgrades for access to new physics  
Highly capable operations team supports expanded user group

*Provide exciting opportunities and stimulating work environment to recruit and train next generation fusion scientists*



# Key DIII-D Program Goals Can Motivate a Vibrant and Expanding US Fusion Program With an Energy Goal

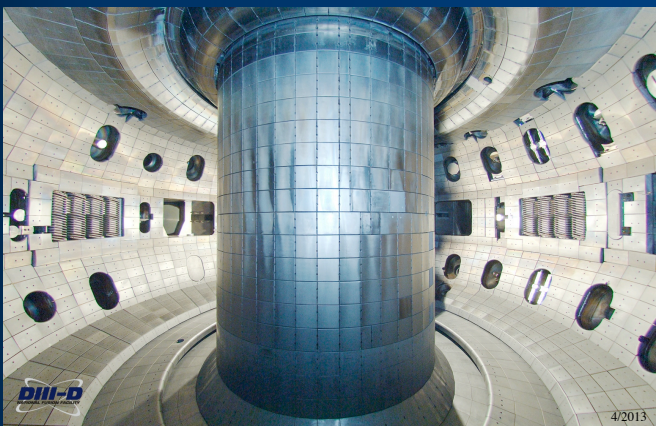


**Enabled by a highly capable facility with technical reach and flexibility to probe the relevant physics of burning plasmas**

# Comprehensive Diagnostics Provide a Strong Foundation to Advance Understanding Through Integrated Simulation

## DIII-D Has a Comprehensive Diagnostic Set in Support of Its Scientific Objectives

- Tile current array
- FIDA
- Bolometers
- IR cameras
- Fast ion collectors
- SXR
- Filterscopes
- MSE
- FIR &  $\mu$ w scattering
- BES
- SPRED
- Vertical scanning probe
- Magnetics
- ECEI & MIR
- MDS spectrometer
- Visible bremsstrahlung
- Gamma detectors
- Fast wave reflectometer
- Coherence Imaging
- Lithium beam spectroscopy
- Neutrons
- Thermocouples
- Phase contrast
- Radial scanning
- Thomson scattering
- CP swing probes
- Langmuir probes
- DISRAD
- UF-CHERS
- CECE
- CER
- VUV cameras
- ASDEX gauges
- Visible cameras
- Fast framing camera
- DBS
- DiMES
- ECE
- NPAs



www.physicstoday.org  
**physics today**  
 October 2015  
 A publication of the American Institute of Physics  
 volume 68, number 10

**Hot times for fusion plasmas**

also:  
 Imaging for proton-beam therapy ◀  
 A galaxy in the cosmic web ◀  
 Solids under tension ◀

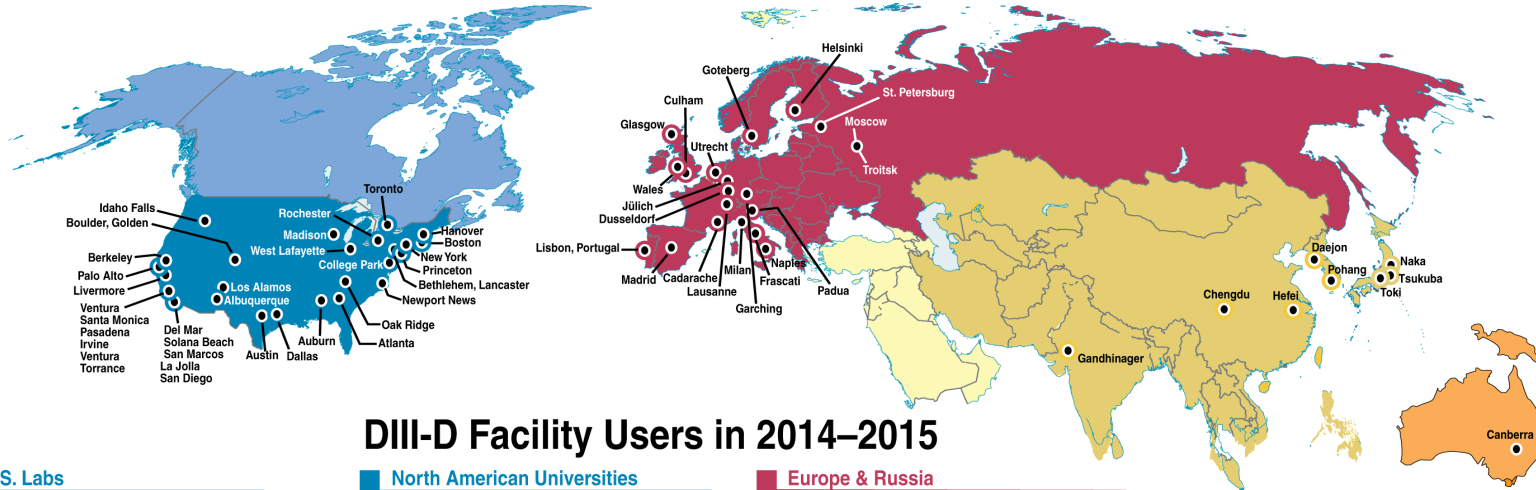
**REPORT OF THE WORKSHOP ON INTEGRATED SIMULATIONS FOR MAGNETIC FUSION ENERGY SCIENCES**  
 JUNE 2 - 4, 2015

Office of Science  
 U.S. DEPARTMENT OF ENERGY

Sponsored by the Office of Fusion Energy Sciences and the Office of Advanced Scientific Computing Research

**New predictive capabilities**

# Integrated International Team With Diverse Capabilities Is the Key Strength of the Program



## DIII-D Facility Users in 2014–2015

### U.S. Labs

Idaho National Laboratory  
 Jefferson Lab  
 Lawrence Berkeley National Laboratory  
 Lawrence Livermore National Laboratory  
 Oak Ridge National Lab  
 Princeton Plasma Physics Laboratory  
 Sandia National Laboratory

### U.S. Industries

American Physical Society  
 Beach Access Software (San Diego)  
 CompX (San Diego)  
 Eagle Harbor Technologies, Inc.  
 Far-Tech, Inc. (San Diego)  
 Fourth State Research (Austin)  
 General Atomics (San Diego)  
 IMSOL-X (San Diego)  
 Kalling Software (New York)  
 Tech-X Corporation (Boulder)  
 Tri Alpha Energy, Inc.

### U.S. Academic Institutions

American Physical Society  
 Oak Ridge Institute for Science Education

### South America

Centro Atomico Bariloche (Argentina)  
 University of Sao Paulo (Brazil)

### North American Universities

Auburn University  
 Carnegie Mellon University  
 Columbia University  
 Georgia Tech (Atlanta)  
 Horizon Prep (San Diego)  
 Lehigh University  
 Massachusetts Institute of Technology  
 Oak Ridge Associated Universities  
 Palomar College  
 Princeton University  
 The College of William and Mary  
 University of Arizona  
 UC Berkeley  
 UC Davis  
 UC Irvine  
 UC Los Angeles  
 UC San Diego  
 University of Colorado, Boulder  
 University of Maryland  
 University of Texas  
 University of Toronto  
 University of Washington  
 University of Wisconsin  
 West Virginia University

### Europe & Russia

Aalto University, Finland  
 CEA Cadarache (France)  
 Chalmers University of Technology (Sweden)  
 Ciemat (Spain)  
 Consorzio RFX (Italy)  
 D-TACQ Solutions Ltd (UK)  
 Eindhoven University (Netherlands)  
 ENEA C.R. Frascati (Italy)  
 EPFL (Lausanne, Switzerland)  
 Forschungszentrum Juelich (Germany)  
 Huazhong University of Science and Technology  
 IFP - Consiglio Nazionale delle Ricerche (Italy)  
 Institute of Control Sciences (Moscow)  
 Institute of Plasma Physics AS CR, Czech Republic  
 Instituto Superior Tecnico, Lisboa, Portugal  
 Istituto di Fisica del Plasma CNR-EURATOM (Italy)  
 ITER Organization  
 Kungliga Tekniska Hogskolan (Stockholm)  
 Max-Planck Institute for Plasma Physics  
 Politecnico di Milano (Italy)  
 RRC Kurchatov Institute  
 Technical University Munich  
 TRINITY lab  
 United Kingdom Atomic Energy Authority (CCFE)  
 Università degli Studi di Padova  
 Università di Napoli Federico II  
 University of Seville  
 University of Strathclyde  
 University of York  
 VTT Technical Research Centre (Finland)

### Asia

ASIPP Hefei, (China)  
 Dalian University of Technology, China  
 Insitute for Plasma Research (India)  
 Ishikawa National College of Technology (Japan)  
 ITER-India  
 Japan Atomic Energy Agency  
 KAIST (Korea)  
 Korea National Fusion Research Center  
 METU - Middle East Technical University (Turkey)  
 National Fusion Research Institute (Korea)  
 National Institute for Fusion Science, Japan  
 Peking University  
 Seoul National University  
 Southwestern Institute of Physics, China  
 Tohoku University  
 USTC (Hefei, China)

### Australia

Australian National University (Sydney)

- 557 Users
- 32 Countries
- 95 Institutions
- 63 Grad Student Users
- 54 Post Doc Users

# DIII-D High-Level Research Objectives Are Well Aligned With Restructured DOE-FES Program

## DIII-D Research Objectives

### 1. Prepare for Burning Plasmas

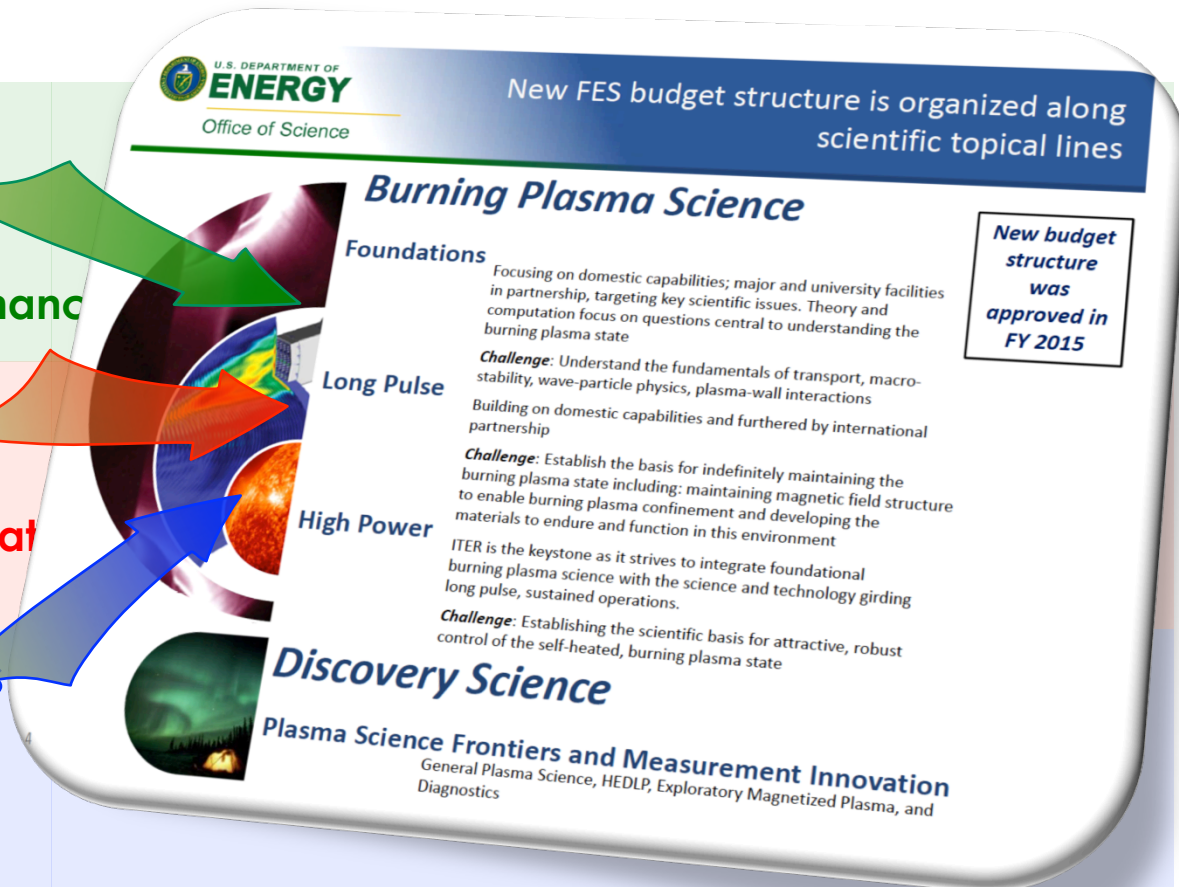
Deliver predictive understanding of the impact & optimization of burning plasma conditions on plasma performance

### 2. Determine Path to Steady State

Provide requirements for achieving efficient, high performance, steady-state tokamak operation

### 3. Develop PMI-Boundary Solutions

Develop and validate solutions for heat flux control including transients in ITER and future devices



DIII-D Funding provided under Foundations

# DIII-D Program Research Objectives Are Well Aligned With Recent Community Workshop Initiatives

## DIII-D Research Objectives

- 1. Prepare for Burning Plasmas**  
Deliver predictive understanding of the impact & optimization of burning plasma conditions on plasma performance
- 2. Determine Path to Steady State**  
Provide requirements for achieving efficient, high performance, steady-state tokamak operation
- 3. Develop PMI-Boundary Solutions**  
Develop and validate solutions for heat flux control including transients in ITER and future devices

## Initiatives from Planning Workshops

Integrated Simulation Initiatives

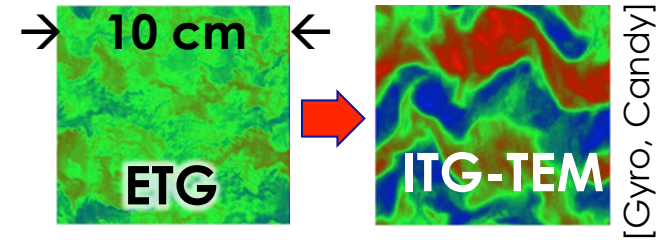
Transients Initiatives

PMI Solutions Initiatives

- **Broad engagement by DIII-D and GA theory programs**
  - Over 40 scientists on panels
  - Chair, co-chair of Transients Workshop, working group chairs
- **Workshops: diverse, lively discussions**

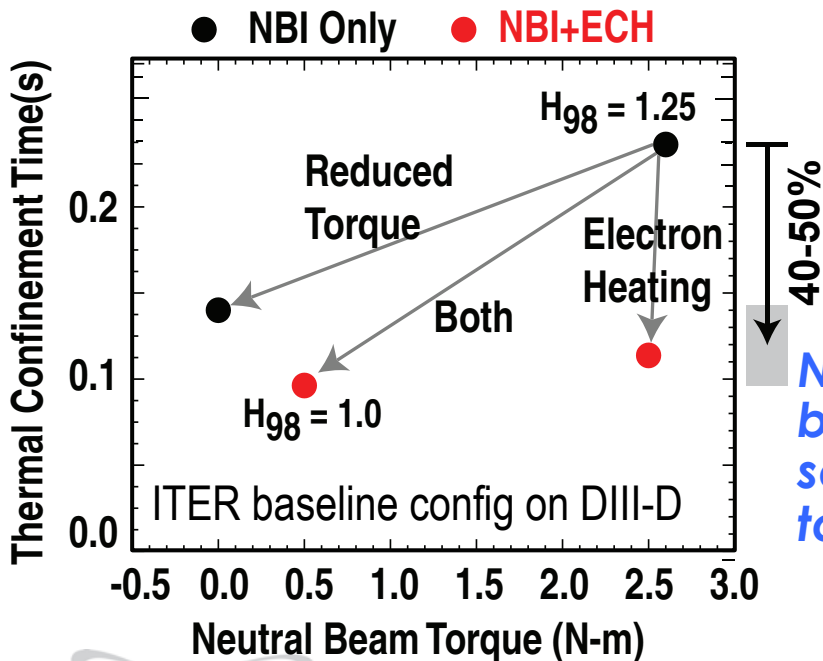
# Planned Heating & Current Drive Upgrades Advance Transport Studies To Reactor-Like Conditions

- **Challenge:** Theory and experiment show transport will change in burning plasmas
  - Turbulence altered with stronger electron heating and reduced flow shear

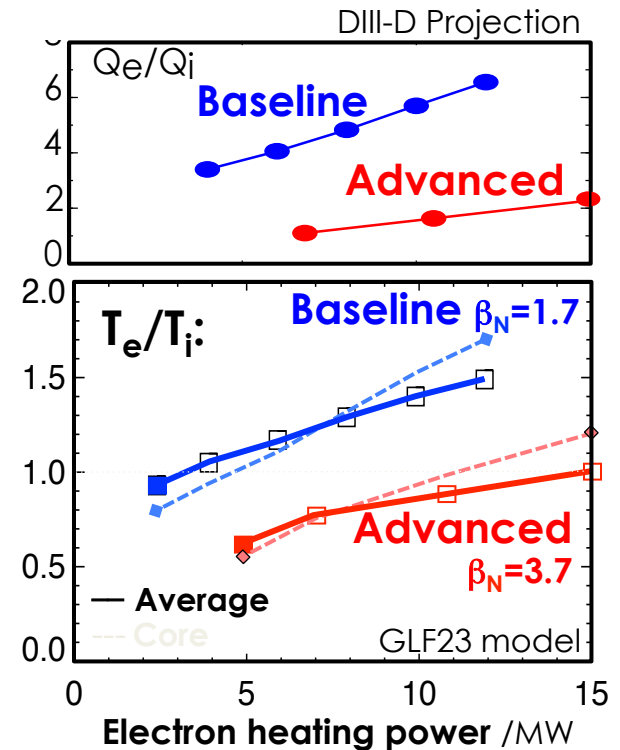


- **DIII-D will access dominantly electron heated low rotation regimes to understand turbulence properties over wide range in  $\beta$**

- Increase gyrotron power and toroidally steer neutral beams to triple range accessed



*Need to develop basis for improve scenarios with low torque e<sup>-</sup> heating*

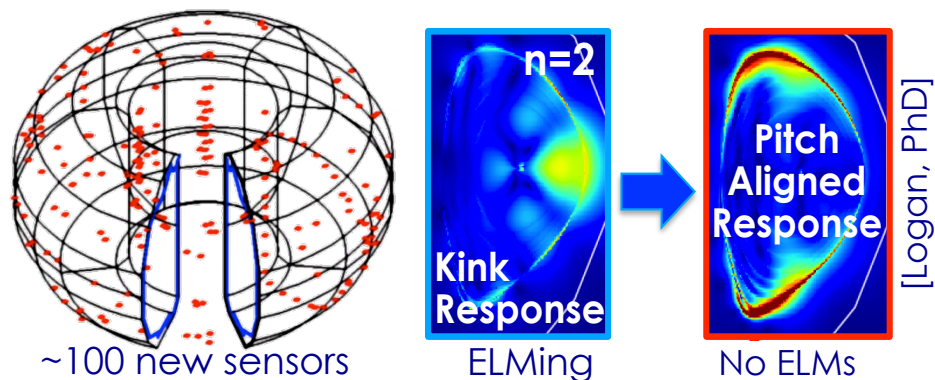


**Simulation Initiative**



# DIII-D is Discovering Physics Underlying ELM Suppression to Move Beyond Demonstration Experiments

## 3D fields resonate with plasma to stop ELMs



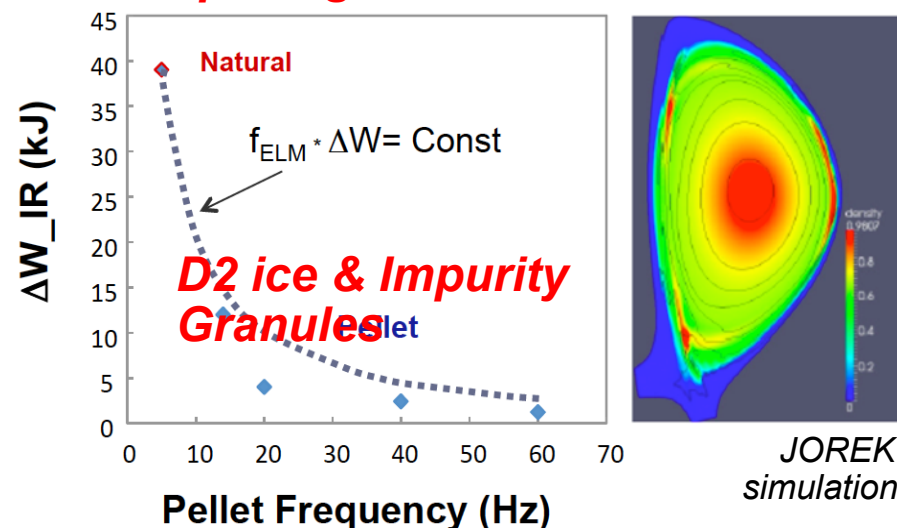
- **Address extrapolation issues**

- Raise 3D flexibility to isolate spectral features for ELM & stability control
- Pellet ELM triggering mechanisms

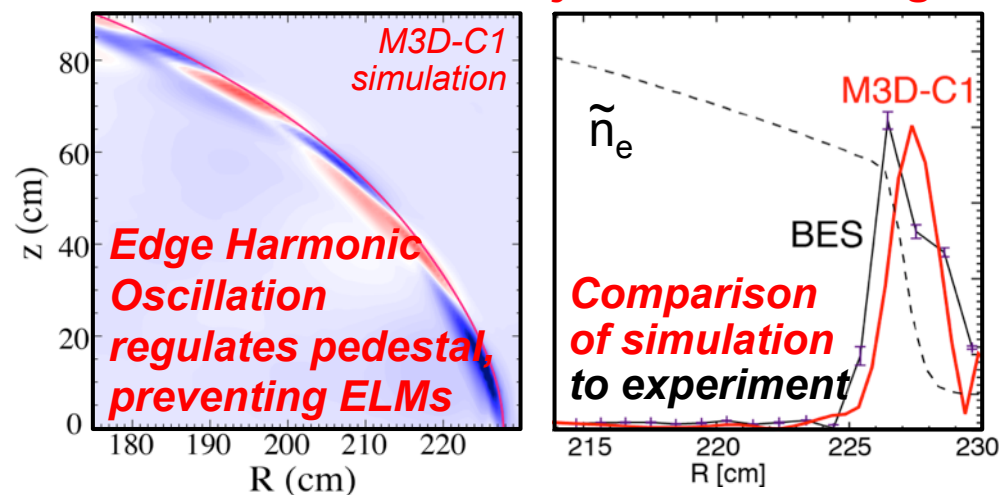
- **Produce ELM-free regimes under reactor-relevant conditions**

- Utilize “3D” super-supplies (ASIPP China), possible new set of 3D coils to optimize spectrum

## Pellet pacing reduces ELM heat loads



## QH mode: an inherently ELM-stable regime



Transients and Simulation Initiatives

# Meeting the Disruption Challenge: DIII-D Will Resolve the Physics for Safe Quenching of Tokamak Plasmas

- U.S. responsible for ITER disruption mitigation system

- Energetic runaway electrons
- Localized heat loads & forces

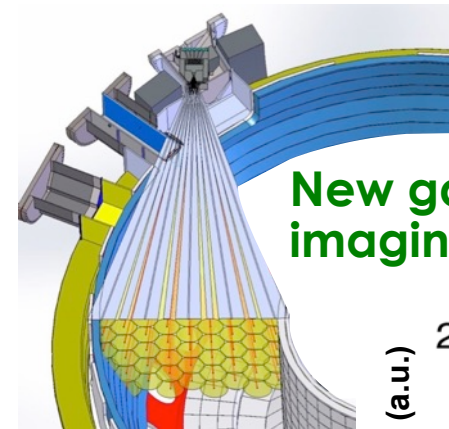
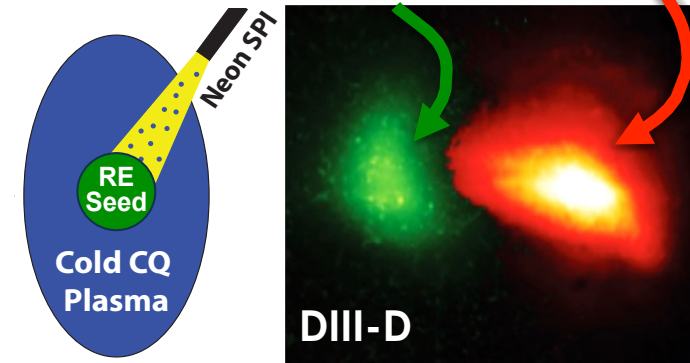
- DIII-D research seeks to

- Resolve physics of runaway dissipation
- Understand radiative asymmetries
- Optimize mitigation schemes
- Compare with non-linear models and theory for reliable projection

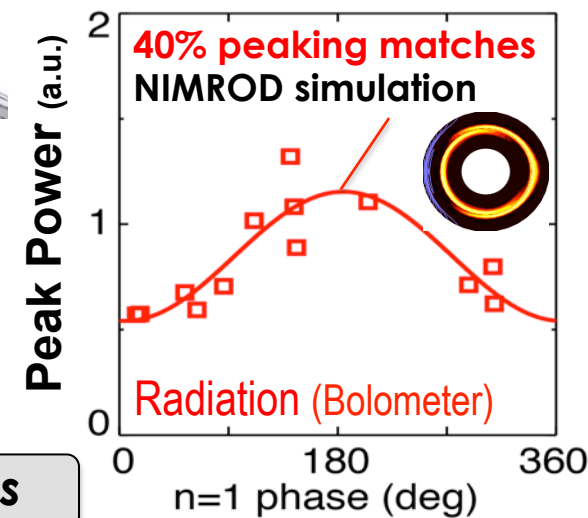
- Utilize 3D diagnostics & injector developments

**Basis for robust safe termination of plasmas in ITER & beyond**

Neon shattered pellet impacts runaway electrons



New gamma ray imaging camera



Transients and Simulation Initiatives

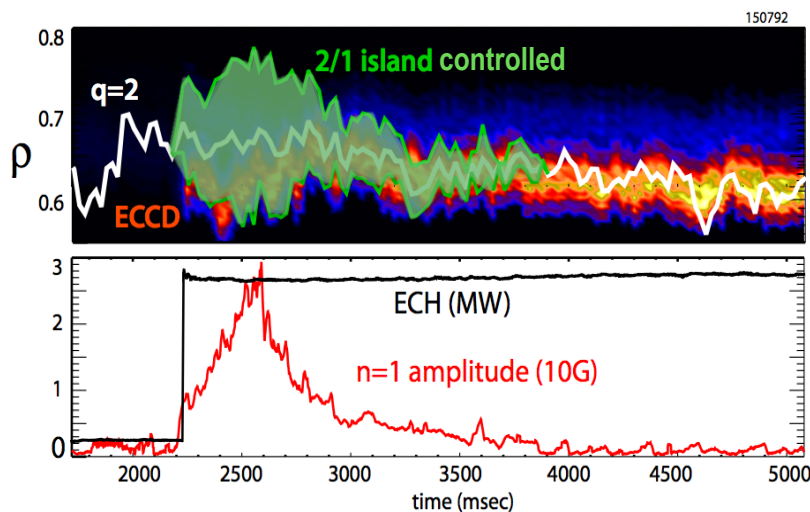
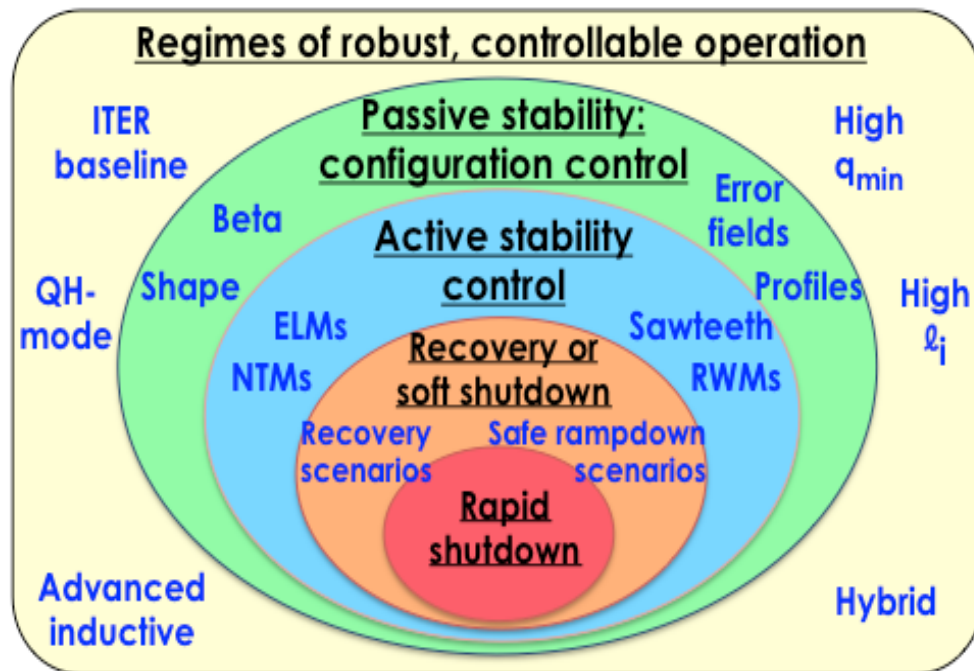
# Research Will Develop a Multi-layered Approach to Achieve Robust Reliable Operation

- **Research will**
  - **Understand instabilities** and how to predict or sense their onset
  - **Develop key actuators** and project to future devices
  - **Resolve integrated control strategies** in relevant scenarios

**Increase EC & balanced-NBI power**  
**Increase 3D flexibility to access key regimes, do perturbative studies**

*Significant progress in proof of principle control methods →*

**Basis for reliable control of burning plasmas**



**Transients, Simulation Initiatives**

# A Steady State Burning Plasma Requires Both High Plasma Pressure and Self-Driven Plasma Current

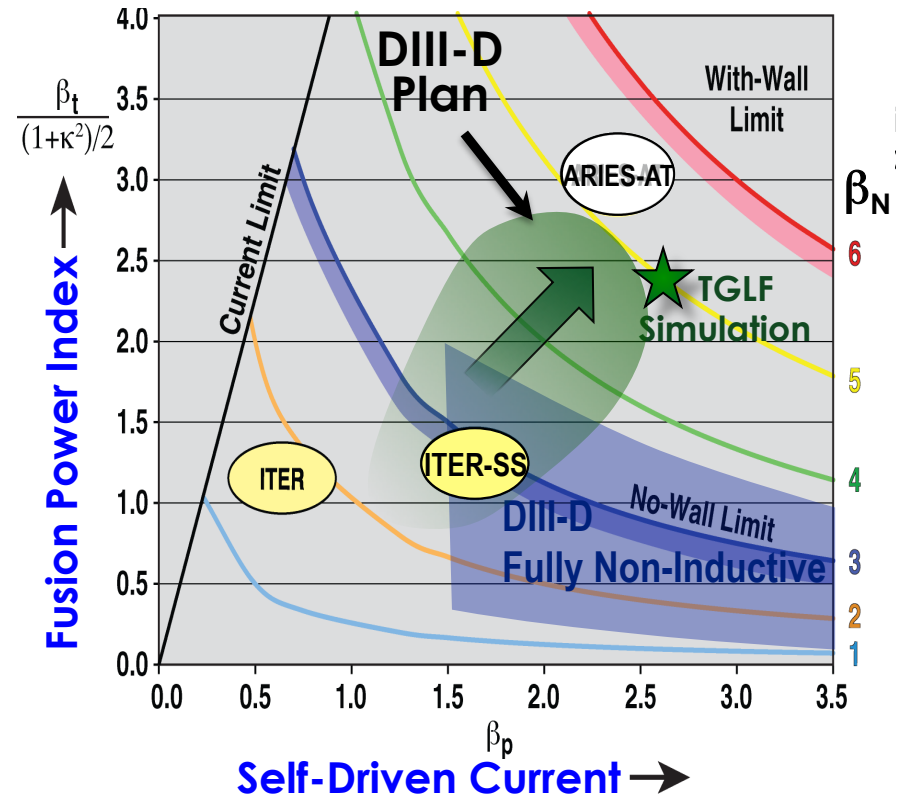
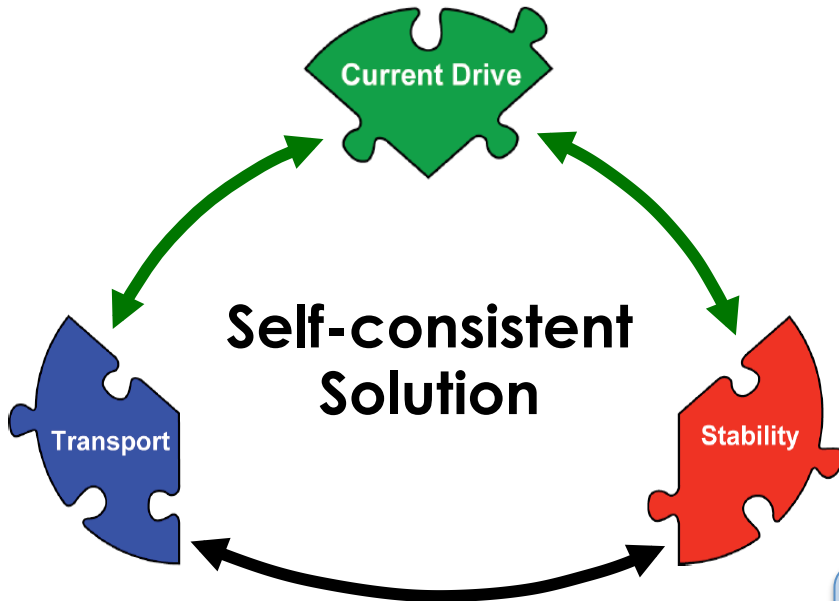
## Goal

**High pressure + High self-driven current**

**Fusion power      Steady-state & high energy gain**

**Must explore physics at high  $\beta_N$  to determine path for future reactors**

- Increase heating power and profile flexibility through beams and ECH



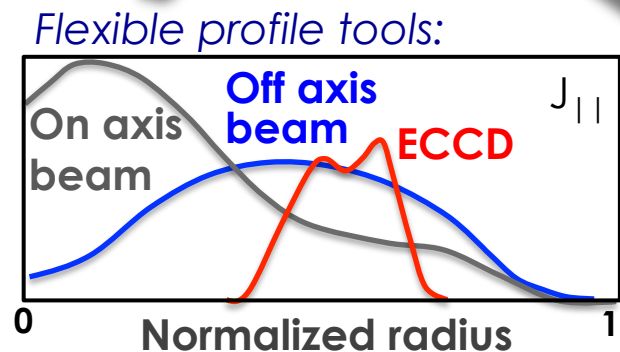
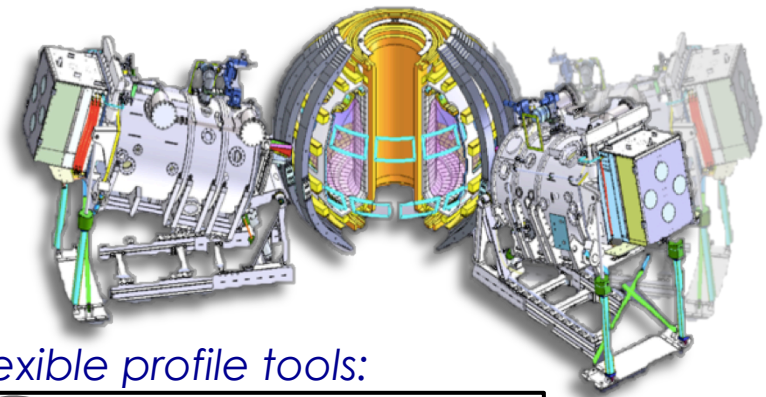
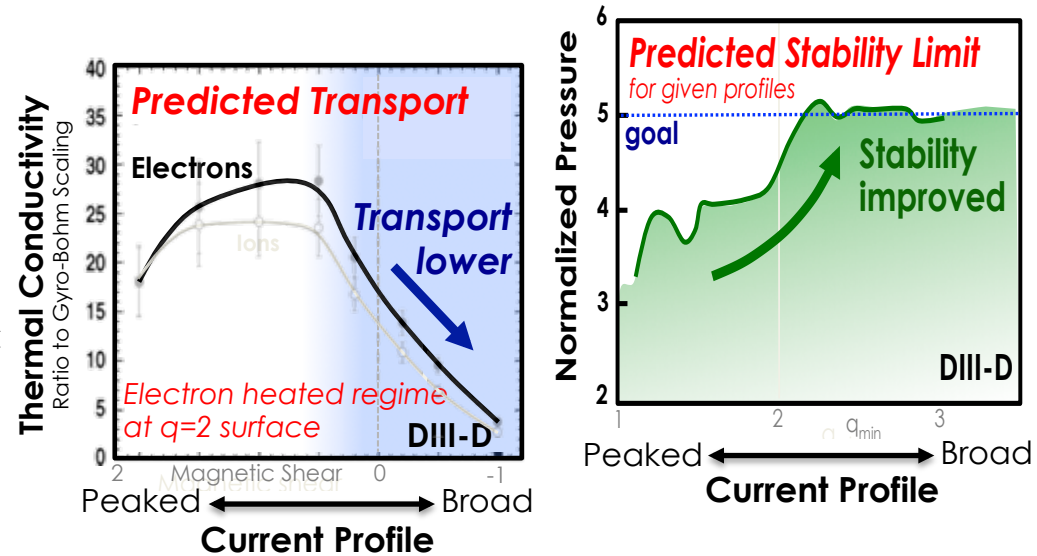
- **Tight coupling between profiles, transport and stability**
  - Self-consistent scenario required
  - Potential solutions with broad or peaked profiles

**Higher  $\beta_N$  enables a more compact and cost-effective reactor**

# Profile Flexibility Will Enable DIII-D to Study the Key Physics at High $\beta_N$ For Reactor Solutions

- **Research will explore key profile dependencies in high  $\beta$  regime**
  - Stability above the no-wall limit
  - Turbulence modification by magnetic shear
  - Fast ion redistribution physics
- **Enabled by 2.5x off-axis current drive and 3x electron heating:**
  - 2<sup>nd</sup> off axis beam & increased ECH
  - Toroidally steerable beams to raise co- and balanced-torque power

**Will determine scientific foundations and existence proofs for viable steady state**



**Simulation, Transients Initiatives**

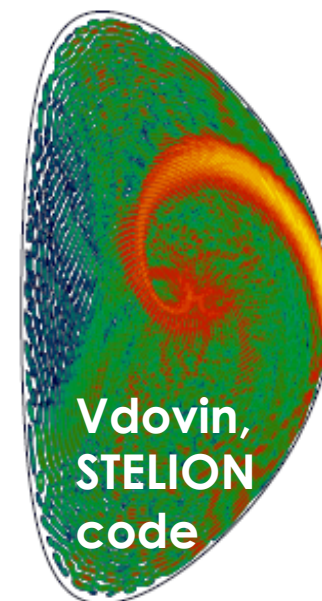
# Helicon Implementation Progressing Well and On Track for Key Tests as a Transformational Current Drive Source

**Goal:** Test Very High Harmonic Fast Wave (~500 MHz) as an efficient off-axis CD technique for future fusion devices

*DIII-D Helicon deposition validated by 3 codes:*

## Context: Helicon & DIII-D potential

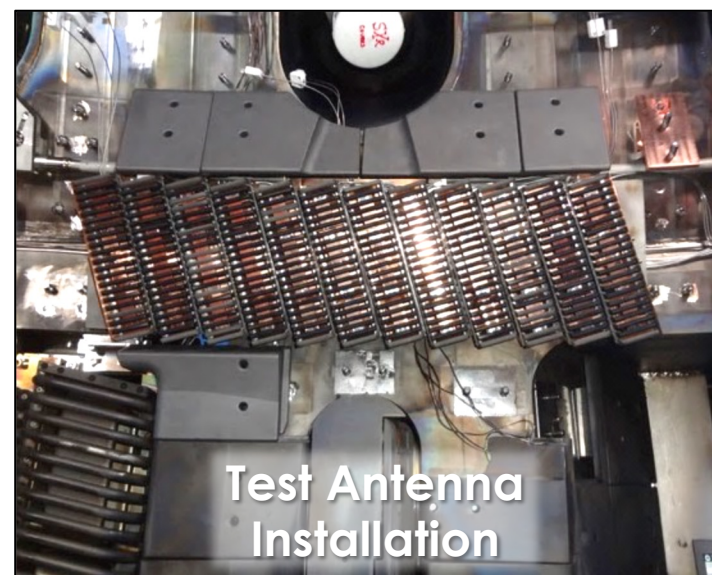
- 2-4x more efficient than ECCD or NBCD
- Comb-line antennas a tested technology
- Requires high  $\beta_e$  and current drive measurement



## Research Plan

- Test prototype antenna (100 W) in fall 2015
- Install and test 1MW system in early 2017
- Compare with simulation and assess current drive efficiencies

**Klystrons supplied from  
SLAC at modest cost**



# Advanced Divertors Minimize and Simplify the Volume Needed for Reliable Dissipation of Plasma Losses

## 1. Advance physics understanding to develop improved divertor concepts

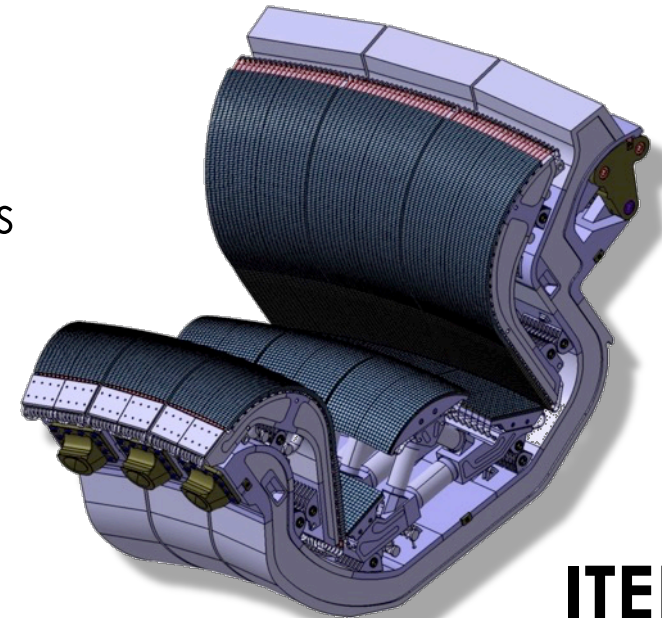
- Enhanced measurements to isolate physics
- Systematic divertor modifications to determine key dependencies
- Compare against state-of-the-art numerical simulation

## 2. Test Advanced Materials for fusion

- Evaluate erosion, migration and re-deposition on a variety of scales
- Assess impact and mitigate impacts on core plasma

## 3. Assess compatibility with high performance core

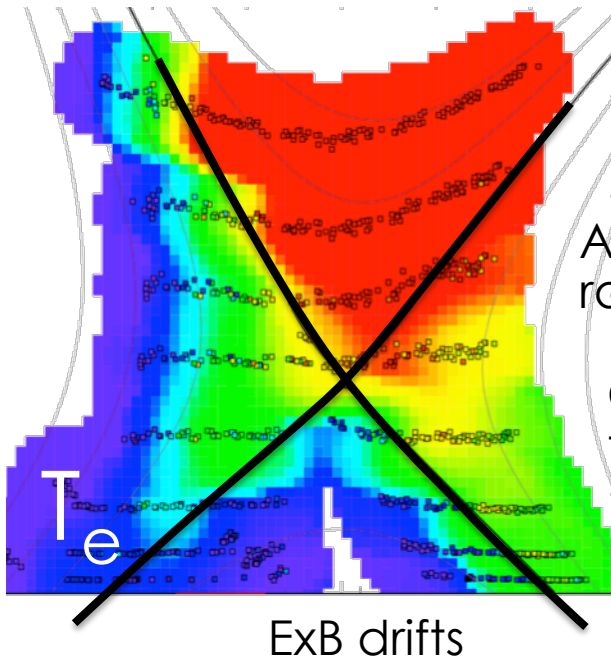
**Validated models are essential for extrapolating to future devices and to design optimized divertors**



**ITER  
divertor module**

# Developing Power Handling Solution Requires Comprehensive Understanding of Detachment Physics

## Complex, dynamic multi-scale physics

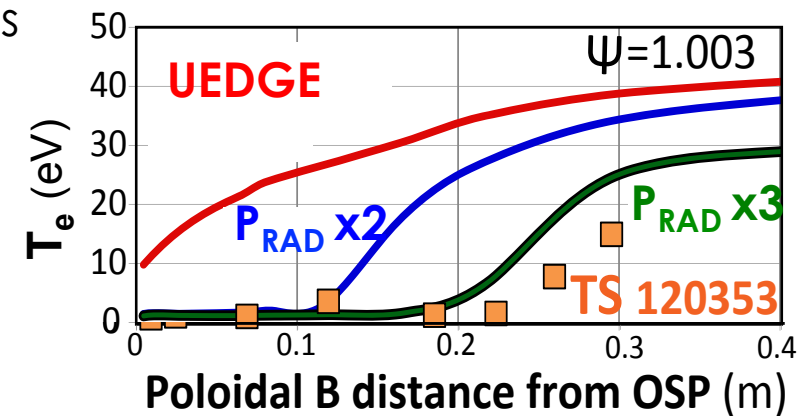
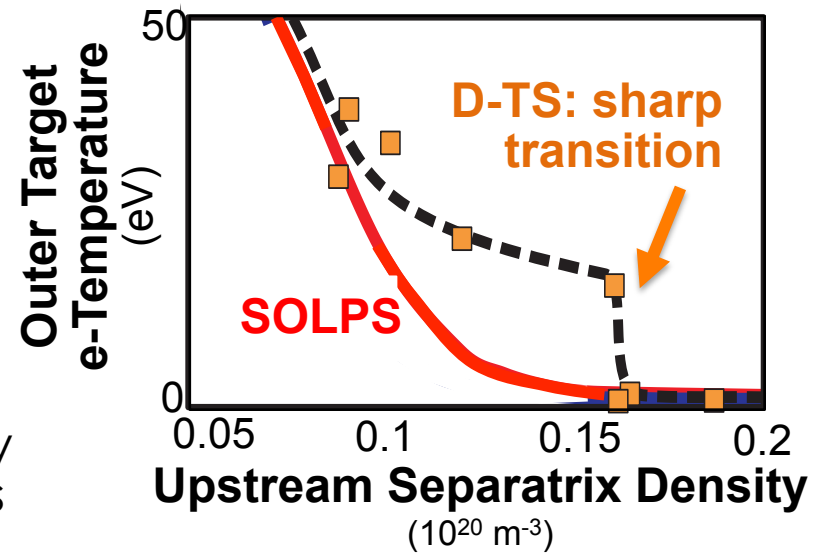


SOL turbulent transport, drift, and kinetic effects

Atomic & molecular radiation & recombination

Charge exchange, impurity transport, neutral dynamics

Sheath, surface interactions (sputtering, recycling...)



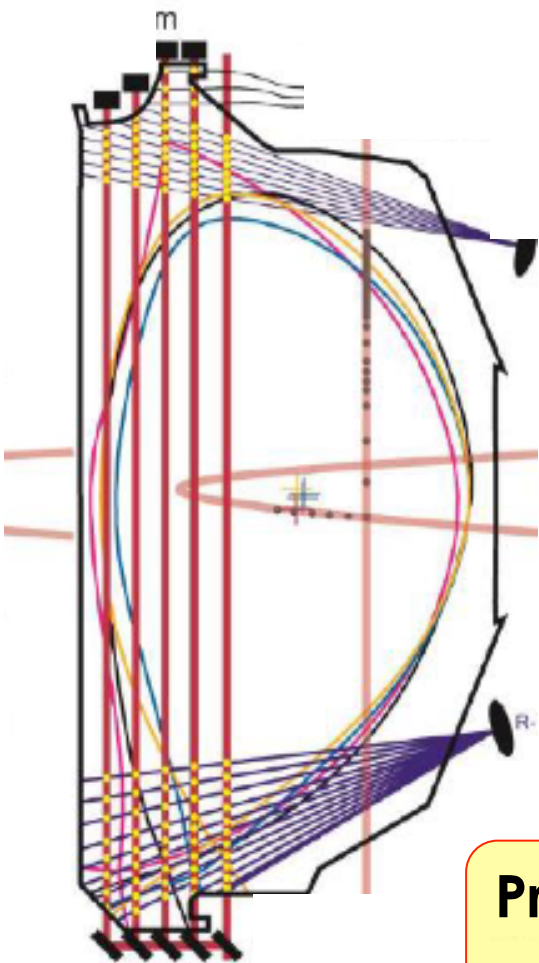
**Codes must reproduce non-linearities and are key to providing predictive understanding for divertor optimization**

*Simulation and PMI Initiatives*



# Advancing Physics Understanding Requires Improved Diagnostics and Systematic Variation of Configuration

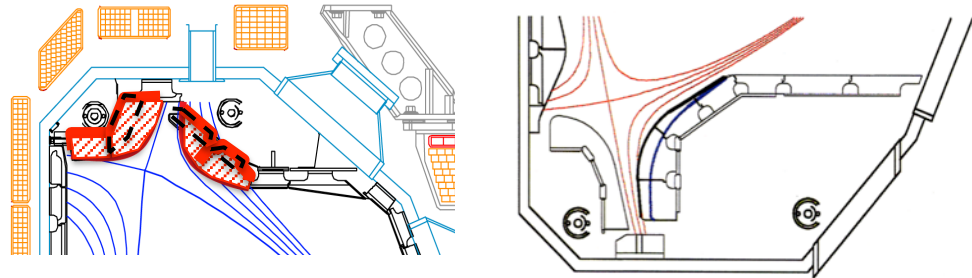
## Upper & lower 2D Thomson scattering



## Further enhancements

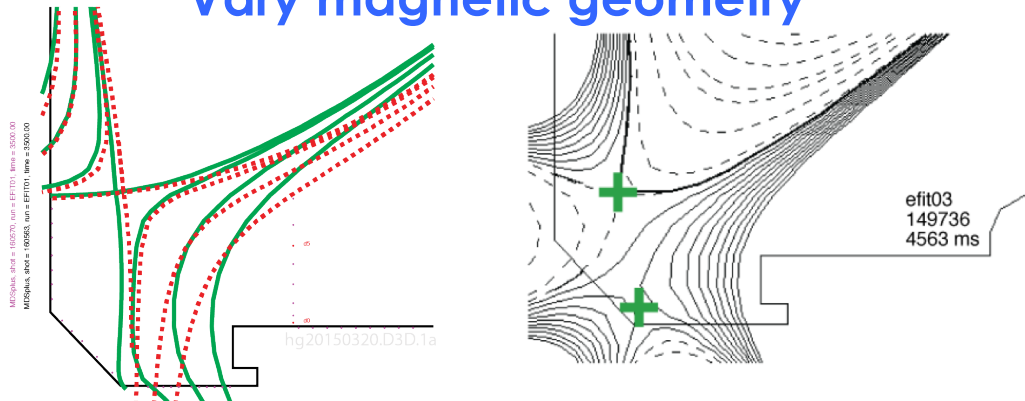
- Bolometry,
- Spectroscopy,
- Neutral pressure,
- Ion Temperature

## Systematically vary closure



**FY17 Joint Research Target**

## Vary magnetic geometry



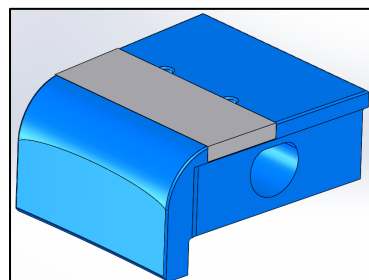
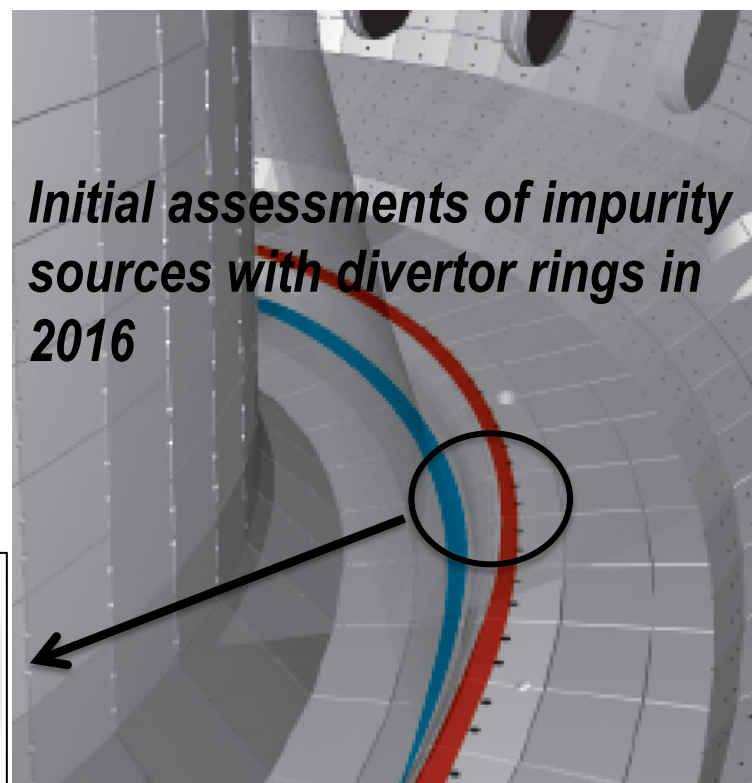
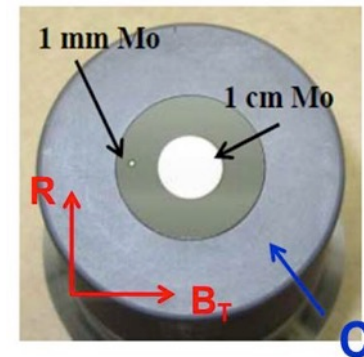
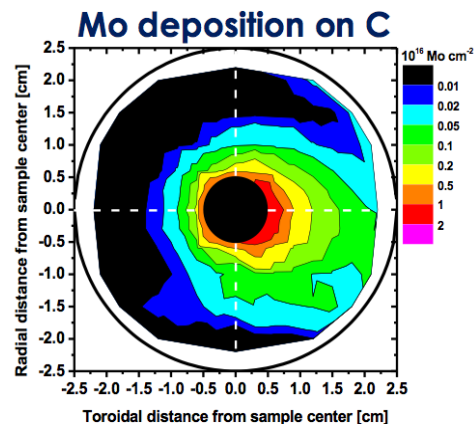
Flaring, flux expansion, connection length, additional X points

**Provides basis for future optimized divertor to be tested in DIII-D**

**Simulation and PMI Initiatives**

# DIII-D Provides Unique Capability to Validate PMI in Reactor-Relevant Tokamak Environment

- **Test emergent materials in fusion relevant plasmas**
  - Assess physics of erosion, re-deposition & migration
  - Measure temperature dependence
  - Evaluate surface evolution
- **Assess impact on plasma performance from large scale divertor PMI**
  - High-Z core contamination & benefits of divertor optimization
  - Effects of high temperature PFCs on core and divertor operation



# Planned Upgrades Will Provide World-Class Capabilities and Flexibility for Addressing Key Scientific Issues

## DIII-D Initiatives



## Enabled by DIII-D Upgrades

### 1. Prepare for Burning Plasmas

Deliver predictive understanding of the impact & optimization of burning plasma conditions on plasma performance



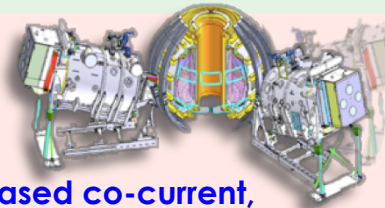
Increased electron Heating/current drive

Super-SPA supply (ASIPP)



### 2. Determine Path to Steady State

Provide requirements for achieving efficient, high performance, steady-state tokamak operation



Increased co-current, off-axis current drive

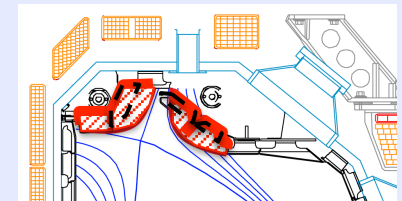
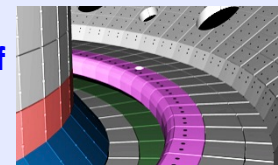
New helicon system



### 3. Develop PMI-Boundary Solutions

Develop and validate solutions for heat flux control including transients in FNSF and future devices

Toroidal rows of metal-coated tiles



Divertor Closure Modifications

**Strong DIII-D program enables synergies with NSTX-U, long-pulse facilities, ITER, university programs, theory community, and diagnostic development**

# Continued Investment in DIII-D Provides a World-Leading Facility for U.S. Scientists to Pursue Fusion Energy Research

- **Leverages \$1B investment in existing world-class facility**

- Extensive, flexible control tools
- Comprehensive diagnostic set



- **Delivers new capabilities that can transform the landscape of fusion science**

- Burning plasma transport
- Self-consistent high  $\beta$  steady states
- Reactor-relevant detached divertor with transients eliminated



- **Provides the foundations for success in U.S. next step devices**

- Burning plasmas in ITER
- Long-pulse, high performance operation in FNSF



**A world-class U.S. fusion user facility providing exciting research opportunities to scientists worldwide**

# DIII-D is a Highly Flexible Facility to Develop Scientific Basis for Optimizing Tokamak Approach to Fusion Energy

DIII-D

- **Vary, rotation, rotational shear, and pressure profiles ( $\beta$ )**
  - 20MW Co+Cntr, On/Off-axis NBI, ECH
- **Current profile control (2-3  $\tau_R$  pulse lengths)**
  - Co/Cntr, On/off-axis NBI, ECCD, Helicon test
- **Vary local gradients,  $T_e/T_i$  (steady & perturb)**
  - ECH/ECCD, On/Off-axis NBI
- **Wide range in density and collisionality ( $\nu^*$ )**
  - Controllable divertor exhaust, low-Z wall
- **Shaping flexibility (ITER, DEMO, exploratory)**
  - 18 shaping coils, PCS, Upr/Lwr divertors
- **Broad range of Transient control tools**
  - 3D coils & diags, ECCD, IGI, pellets, SGI, MGI
- **Systematic divertor modification, diagnosis, modeling**
  - Upr/Lwr; variable config, geom, closure; cryopump
- **PMI: Controlled exposure over a range of scales/times**
  - DiMES, MiMES, metal rings, heated samples

